

Characterization prior to decommissioning

> Structures









Characterization prior to decommissioning

> Stored items





Dose rate		
[mSv/h]		
285		
540		
565		
525		
430		
260		
120		
37		
15		
12		
10		
4		
0.425		
0.135		
0.050		



Waste items







> Installations

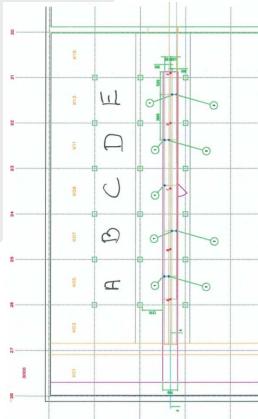


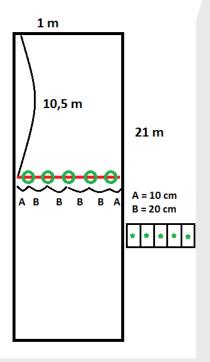




Buildings









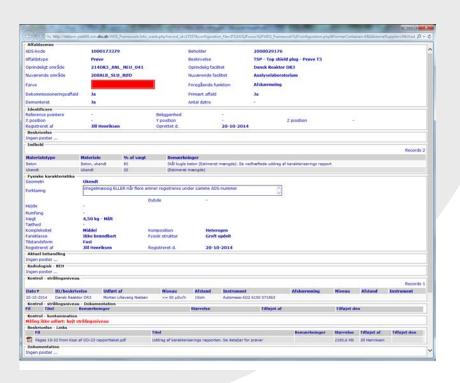
➤ Release of Cl-36?



- Paper sample
- Water sample
- Grass samples



Characterization after decommissioning







Indirect measurements



Sample Measurement Parameters; Comment: EBS sågmotor User: Peter Enqvist Preset Time: 2:00 Alpha Preset Error: 1,0% User Protocol: Strykprov 2 min

Cycle 1 of 1 Start Time: 2015-10-29 11:42:31

Elapsed Time: 2:00 Guard: 13,75 cps

Cycles: 1

Beta Preset Error: 1,0%

Instrument Name: LB 770-2 sn 42361-6236

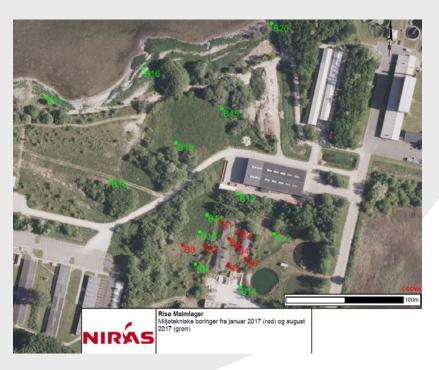
	Spl # Sample Name	Alpha (kBq/m2)	Cat	MDA	MRA	Beta (kBq/m2)	Cat	MDA	MRA
1	11 Motor	0,031 (±74,3%)	1	0,0605	0,0083	11,94 (±4,17%)		0,2189	
2	12 Motor	0,030 (±78,4%)				19,35 (±3,42%)		0,2910	
3	13 Motor	0,030 (±75,7%)	1	0,0638	0,0098	20,45 (±3,32%)		0,2294	
4	14 Motor	0,065 (±51,2%)	1	0,0626	0,0086	8,445 (±4,85%)		0,2413	
5	15 Kabel	$0,049 (\pm 59,5\%)$	1	0,0632	0,0085	0,961 (±15,3%)	1	0,2545	0.1009
6	16 6Kabel	0,000	1	0,0665	0,0100	0,614 (±19,4%)		0,2353	
7	17 7Kabel	0,015 (>100%)	1	0,0664	0,0097	0,895 (±15,8%)		0,2440	
8	18 8Kabel	0,000				0,371 (±26,2%)		0,2240	
9	19 9Kabel	0,000				0,173 (±45,2%)		0,2287	
0	20 Kontackt till kabel	0,000				0,252 (±34,8%)		0,2323	

Tabell 3.4: Ytaktivitet, betastrålare

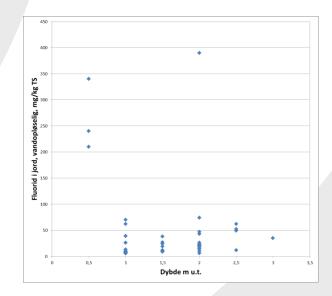
Nuklid	Ytaktivitet, 2 % ²⁴¹ Am	Ytaktivitet, 7,7 % ²⁴¹ Am
	(Bq/m ²)	(Bq/m^2)
⁶⁰ Co	1,11E+06	1,11E+06
¹³⁷ Cs	9,69E+03	9,69E+03
¹⁴ C	7,68E+03	7,68E+03
³⁶ Cl	5,23E+03	5,23E+03
⁵⁵ Fe	2,33E+04	2,33E+04
⁶³ Ni	5,32E+05	5,32E+05
¹⁵⁴ Eu	9,66E+03	9,66E+03
¹⁵⁵ Eu	7,31E+03	7,31E+03
90Sr	6,54E+03	6,54E+03
⁹⁹ Te	1,85E+05	1,85E+05
¹³⁴ Cs	2,98E+02	2,98E+02
²⁴¹ Pu	2,52E+04	9,69E+04
Summa betastrålare	1,9E+06	2,0E+06



Soil sampling



From radiological interest to conventional





Clearance

Clearance is the removal of radioactive materials, buildings (and land areas) from regulatory control, which governs radioactive materials. Clearance is possible if the sum of ratios between activity concentrations (C_i^*) and the radionuclide specific clearance levels (CL_i) is below 1.

$$\sum_{i=1}^{N} \frac{C_i^*}{CL_i} < 1$$

$$\sum_{i=1}^{N} \frac{C_i}{CL_i} + 1.65 \cdot \sqrt{\sum_{i=1}^{N} \frac{u_i(C_i)^2}{CL_i^2}} < 1$$



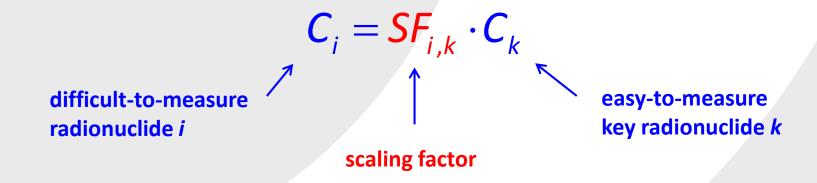
Clearance levels

Radionuclide	Mass-specific clearance levels (IAEA RS-G-1.7) [Bq/g]
⁵⁵ Fe	1000
³ H, ⁴¹ Ca, ⁶³ Ni	100
²³⁶ U, ²⁴¹ Pu	10
¹⁴ C, ⁹⁰ Sr, ¹⁰⁹ Cd, ²³⁴ U, ²³⁵ U, ²³⁸ U	1
⁶⁰ Co, ¹³³ Ba, ¹³⁷ Cs, ¹⁵² Eu, ¹⁵⁴ Eu, All actinides but ²⁴¹ Pu	0.1



Clearance and scaling factors

Measurements on samples can establish conservative activity ratios (scaling factors) between radionuclides which are difficult to measure and a radionuclide (key nuclide) which is more easy to measure.





Clearance and scaling factors

Hot Cell facility: 19 paint samples analysed and conservative activity ratios (scalings factors) extracted. ¹³⁷Cs is the key radionuclide

Nuclide	Scaling factor	Rel. standard uncertainty (%)
⁹⁰ Sr	0.53	30
²³⁸ Pu	0.11	50
²³⁹ Pu	0.054	60
²⁴⁰ Pu	0.041	50
²⁴¹ Pu	1.6	50
²⁴¹ Am	0.093	35
²⁴² Pu	0.00011	50
²⁴³ Cm+ ²⁴⁴ Cm	0.035	50
²⁴² Cm+ ^{242m} Am	0.00077	50



Clearance and scaling factors

Neutron irradiated materials:

$$SF_{i,k} = \frac{m_i \cdot A_k \cdot N_{F,i} \cdot \sigma_i \cdot (1 - e^{\frac{-\ln(2) \cdot \tau}{T_i}}) \cdot e^{\frac{-\ln(2) \cdot t}{T_i}}}{m_k \cdot A_i \cdot N_{F,k} \cdot \sigma_k \cdot (1 - e^{\frac{-\ln(2) \cdot \tau}{T_k}}) \cdot e^{\frac{-\ln(2) \cdot t}{T_k}}}$$

m: relative mass content of mother

A: atomic mass

 N_F : isotopic abundance of mother

 σ : neutron absorption cross section

T: nuclide half-life

 τ : irradiation time

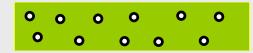
t: time since stop of irradiation

Со	(+/-) 2s Li		(+/-) 2s
ppb ppb		ppb	ppb
4713	1000	22235	2395

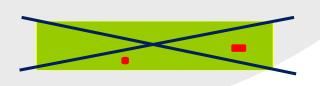




$$\sum_{i=1}^{N} \frac{C_i}{CL_i} + 1.65 \cdot \sqrt{\sum_{i=1}^{N} \frac{u_i(C_i)^2}{CL_i^2}} < 1$$



Statistical test (sign-test) based on measured activity concentrations in samples. N



Clearance by sampling not possible when hot spots.







The outer part of the biological concrete shielding of the DR2 reactor was cleared by measuring on samples from drill cores









Spent fuel storage block iron ball concrete shielding contaminated with ¹⁴C and ³H



14 samples measured, 50 tons cleared



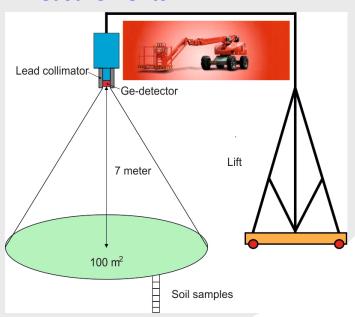








Land area measurements <u>combined with</u> soil measurements







Samples and clearance

Sample material	Radionuclides except γ-emitters	Purpose
Paint	Fission products and actinides	Scaling factors
Paint	³ H, ¹⁴ C	Clearance
Concrete	³H, ¹⁴C, total β	Clearance
Iron	⁶³ Ni, ⁵⁵ Fe, Fe, Co, Ni	Clearance
Paper	U-234, U-235, U-238	Scaling factors
Water	³ H, ¹⁴ C	Clearance



Conclusions

- Sampling is an integral part of all the phases of decommissioning work
- Purpose dictates the sampling method
- Different levels of uncertainty are required
- Radiochemical laboratory is a must for some analyses

